

NURETH-17



Call for Papers

Abstract due : Dec. 15, 2016
Final paper due: Feb. 28, 2017

NURETH-17 Special Topic Special Topic 3: Rod Bundle CHF and Mixing Experiments

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Critical heat flux is one of the key limiting phenomena for the safe operation of nuclear reactors. Recent developments in the design of the fuel assembly, the use of statistical methods, the evaluation of thermal hydraulic conditions as well as the specificities of new types of reactor and passive safety systems bring out new challenges to ensure the first barrier integrity.

We are pleased to invite global experts, scholars and researchers in the field of nuclear reactor thermal-hydraulics to present papers and discuss topics relating to the CHF experiments, mixing tests, and CHF analyses. Please remit your abstract/paper to the following suggested topics or any related subjects ASAP.

Topics include, but are not limited to:

1. Experiments
 - CHF in simple geometries and in rod bundles
 - Mixing in rod bundles
 - Measurement and Instrumentation
 - CHF under transients conditions
 - Forced and Natural Circulation
 2. Code Development Including Numeric Systems
 - Computational Fluid Dynamics
 - Sub-channel Analysis
 - Plant System Code Development
 3. Development of Evaluation Methods
 - Correlation development
 - Treatment of uncertainties
 - Core Thermal Hydraulic Design Methodologies
 - Mechanistic models
 4. New Challenges
 - CHF during blowdown
 - CHF impact due to different cladding surface conditions
 - CHF at low velocity or very low quality conditions
 - SMR related applications
 - Passive Safety Systems and Related Phenomena
 - Abnormal CHF: Flow blockage, rod bow, low pressure low flow, transient CHF, etc.
- Paper submission link: <http://epsr.ans.org/meeting/?m=237>**

